

# **FLOW EFFECTS ON CORROSION OF STEELS IN REACTOR PRIMARY COOLANT – EXPERIENCE AND UNDERSTANDING**

by

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### ▲ CANDU reactors

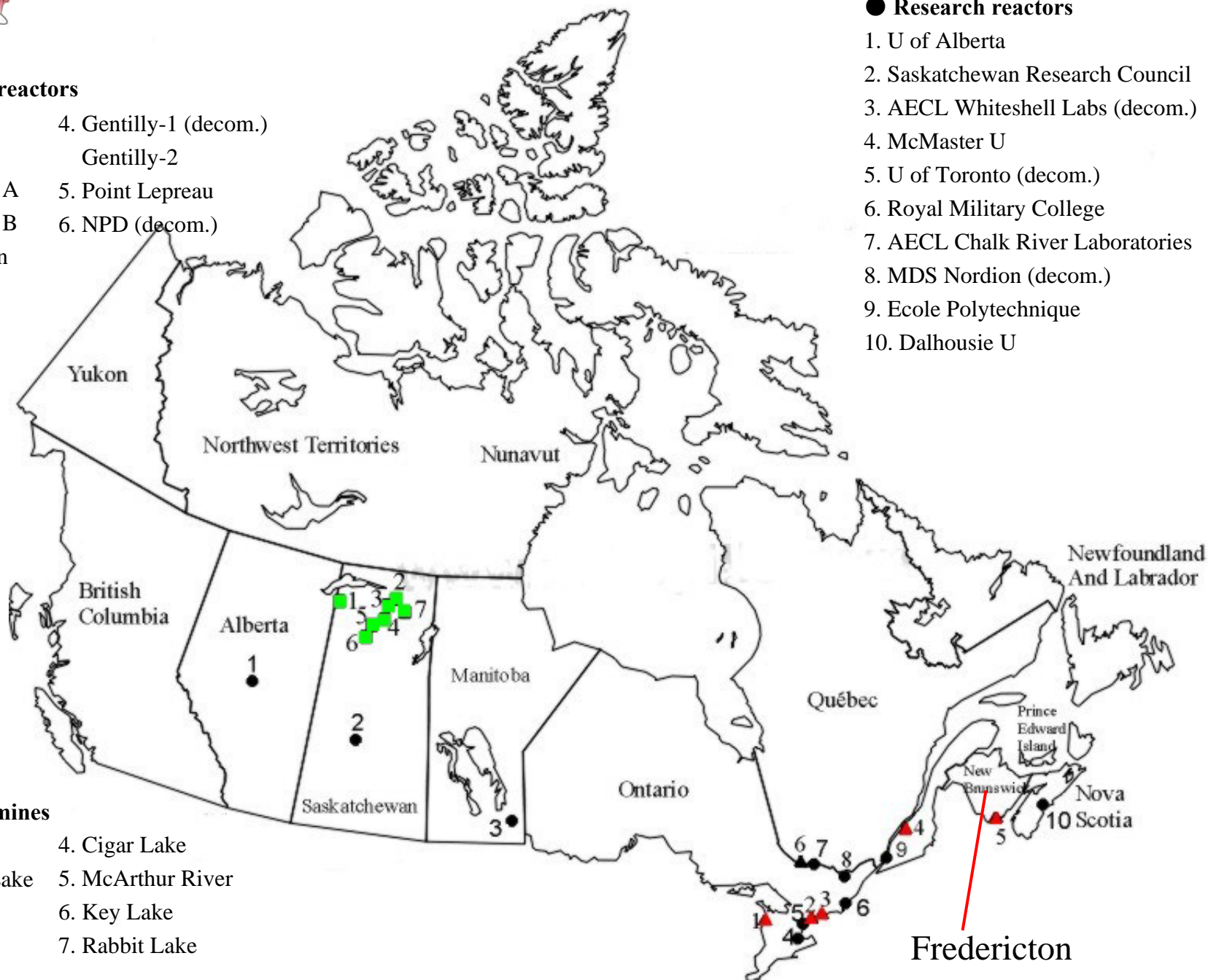
- |                |                        |
|----------------|------------------------|
| 1. Bruce A     | 4. Gentilly-1 (decom.) |
| Bruce B        | Gentilly-2             |
| 2. Pickering A | 5. Point Lepreau       |
| Pickering B    | 6. NPD (decom.)        |
| 3. Darlington  |                        |

### ● Research reactors

1. U of Alberta
2. Saskatchewan Research Council
3. AECL Whiteshell Labs (decom.)
4. McMaster U
5. U of Toronto (decom.)
6. Royal Military College
7. AECL Chalk River Laboratories
8. MDS Nordion (decom.)
9. Ecole Polytechnique
10. Dalhousie U

### ■ Uranium mines

- |                 |                   |
|-----------------|-------------------|
| 1. Cluff Lake   | 4. Cigar Lake     |
| 2. McClean Lake | 5. McArthur River |
| 3. Midwest      | 6. Key Lake       |
|                 | 7. Rabbit Lake    |



## FREDERICTON





## UNB



## **NUCLEAR POWER IN CANADA**

- In 2007, nuclear  $\equiv$  14.6% of Canada's electricity
- Installed nuclear gross capacity in 2008 (all CANDUs):

### **Ontario**

<b>Pickering A</b>	<b><math>4 \times 542</math> MW(e) (two in safe storage)</b>
<b>Pickering B</b>	<b><math>4 \times 540</math> MW(e)</b>
<b>Darlington</b>	<b><math>4 \times 934</math> MW(e)</b>
<b>Bruce A</b>	<b><math>4 \times 805</math> MW(e) (two being refurbished)</b>
<b>Bruce B</b>	<b><math>1 \times 845</math> MW(e)</b>
	<b><math>3 \times 872</math> MW(e)</b>

### **Québec**

<b>Gentilly-2</b>	<b><math>1 \times 675</math> MW(e) (to be refurbished 2011)</b>
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### **New Brunswick**

<b>Point Lepreau</b>	<b><math>1 \times 680</math> MW(e) (being refurbished)</b>
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## **PROSPECTS FOR NEW BUILD IN CANADA**

### **Ontario**

**To decide on technology for two new units in early 2009 (ACR-1000, Areva, Westinghouse)**

### **New Brunswick**

**Negotiating for new unit – Point Lepreau II – ACR-1000 preferred**

### **Alberta and Saskatchewan**

**Interested in new build**

## **CANDU REACTORS ABROAD**

**Korea (Wolsong)      4 × CANDU-6**

**Argentina (Embalse)    1 × CANDU-6**

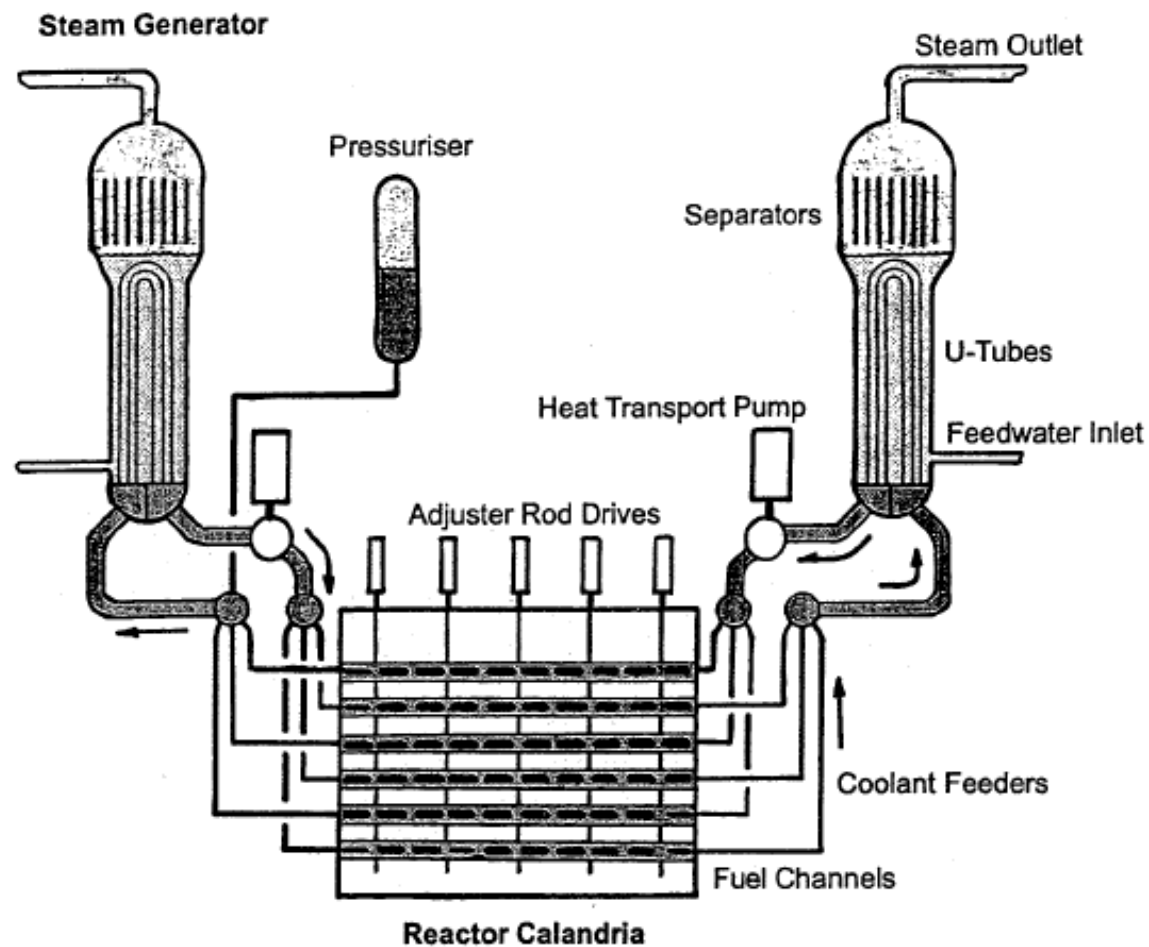
**Romania (Cernavoda) 2 × CANDU-6**

**China (Qinshan)        2 × CANDU-6**

**India                        3 × prototype**

**Pakistan                 1 × prototype**

## CANDU REACTOR CIRCUITS





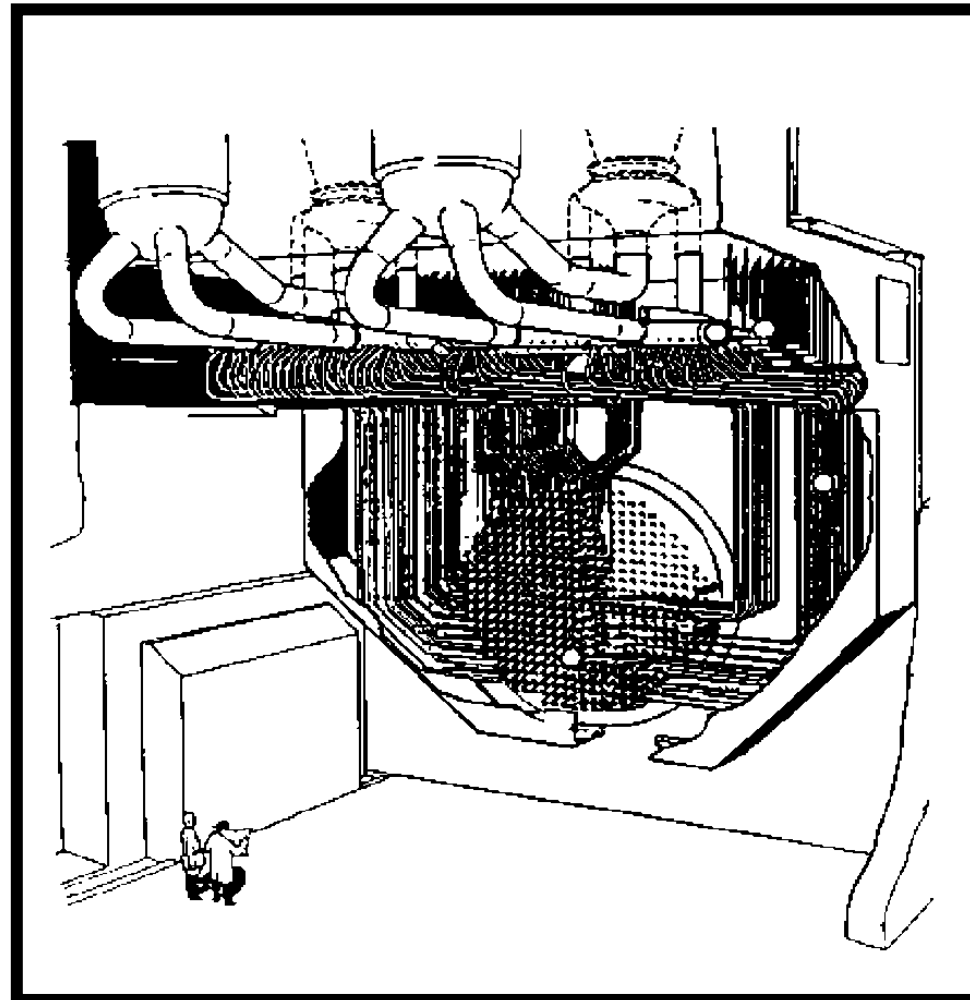
## CANDU PHTS CONDITIONS

- **D<sub>2</sub>O with 3-10 cc/kg dissolved D<sub>2</sub> (H<sub>2</sub> gas added);**
- **Temperature 265-310° C;**
- **pH (apparent) 10.2-10.8 (lithium);**
- **Steam quality at core outlet → 6%;**
- **Dissolved O<sub>2</sub> < 5 ppb (unmeasurable – although coolant may be oxidising at core outlet).**

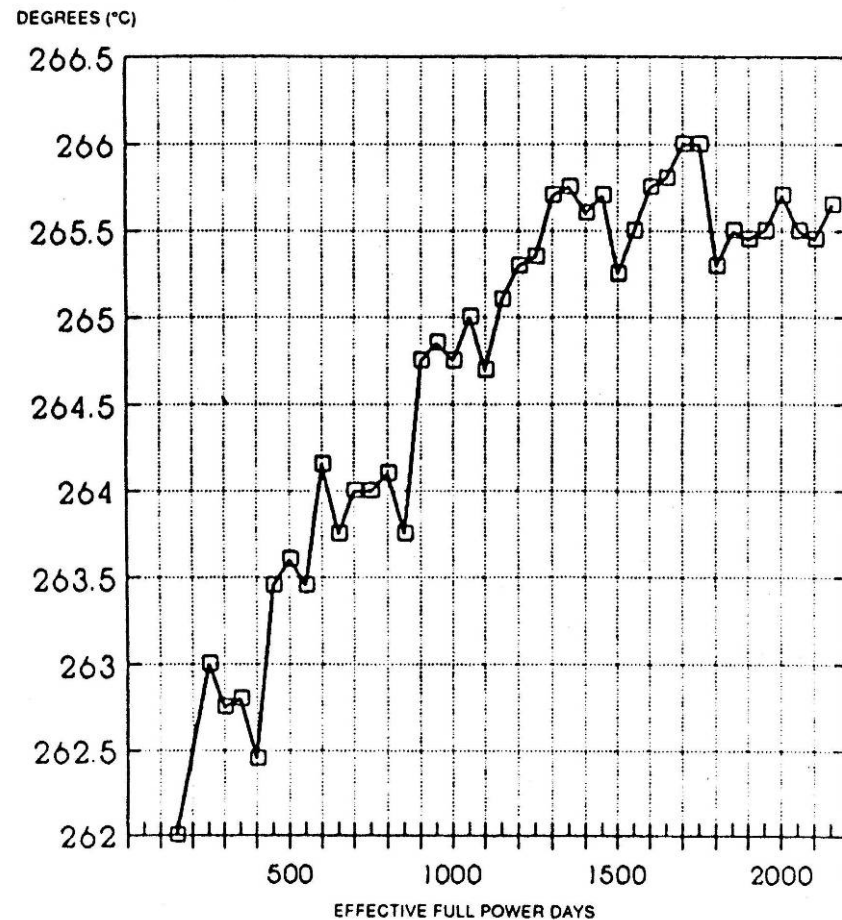
## MAIN PHTS MATERIALS

- **Fuel sheaths – Zircaloy 4**
- **Pressure tubes – Zr-2½%Nb**
- **SG tubing (CANDU 6) – Alloy 800**
- **SG heads – carbon steel**
- **Feeders – carbon steel**
- **Headers – carbon steel**

## CANDU REACTOR FACE



**In 1980s, some CANDUs experienced primary system fouling (reactor inlet temperature rise); e.g. Point Lepreau to May 1989**

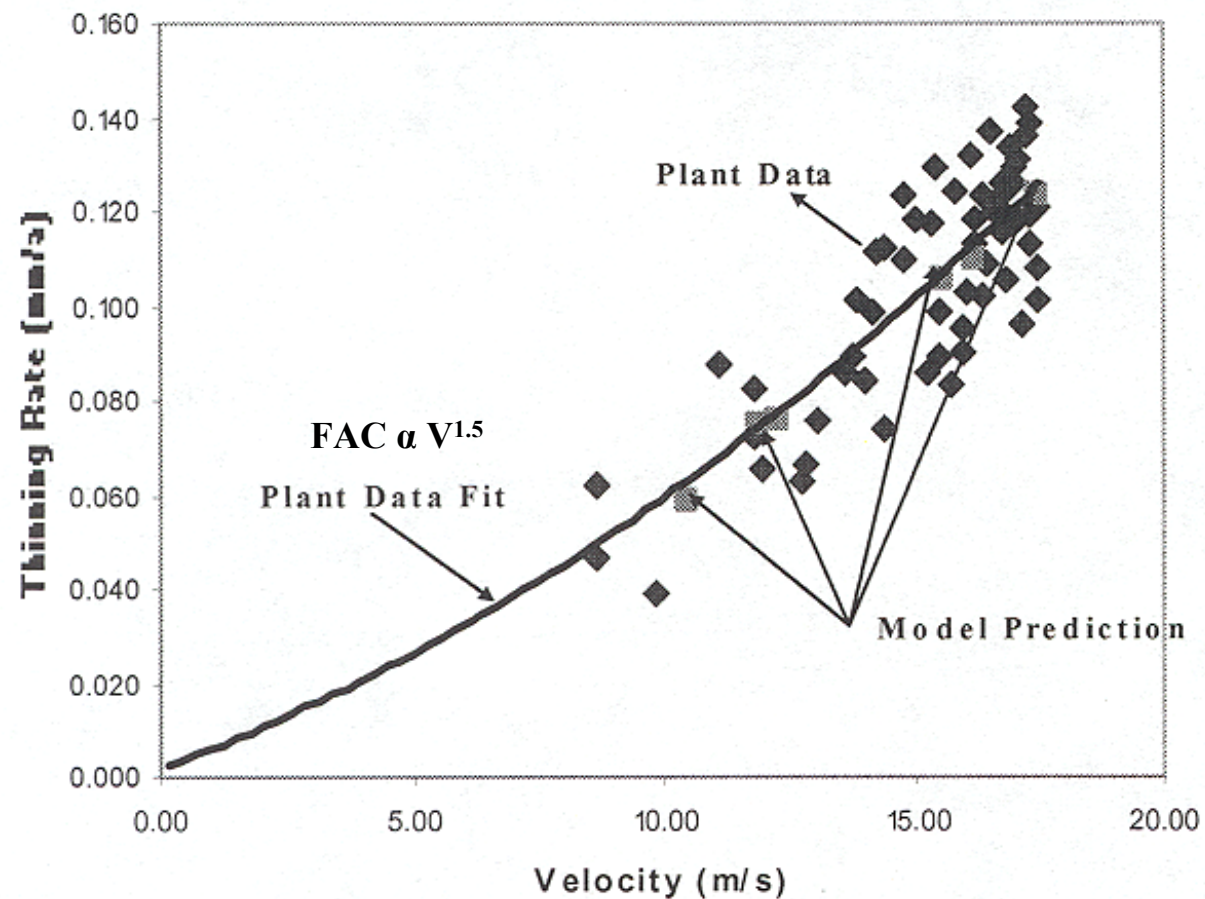




## FEEDER THINNING

**In 1996, ultrasonic measurements of wall thickness of carbon-steel outlet feeders at Point Lepreau indicated excessive corrosion: average rates inferred from estimates of initial thickness.**

## THINNING RELATED TO COOLANT FLOW – FAC (FLOW-ACCELERATED CORROSION)



**In 1997, an outlet feeder at Point Lepreau developed a leak – removed and inspected.**

**Through-wall crack, now thought to be low-temperature creep, possibly exacerbated by hydrogen (deuterium) permeating through metal from FAC.**

**Several cracked feeders found since – one or two through-wall.**

**Only at Point Lepreau.**

## INSIDE SURFACE OF CRACKED FEEDER

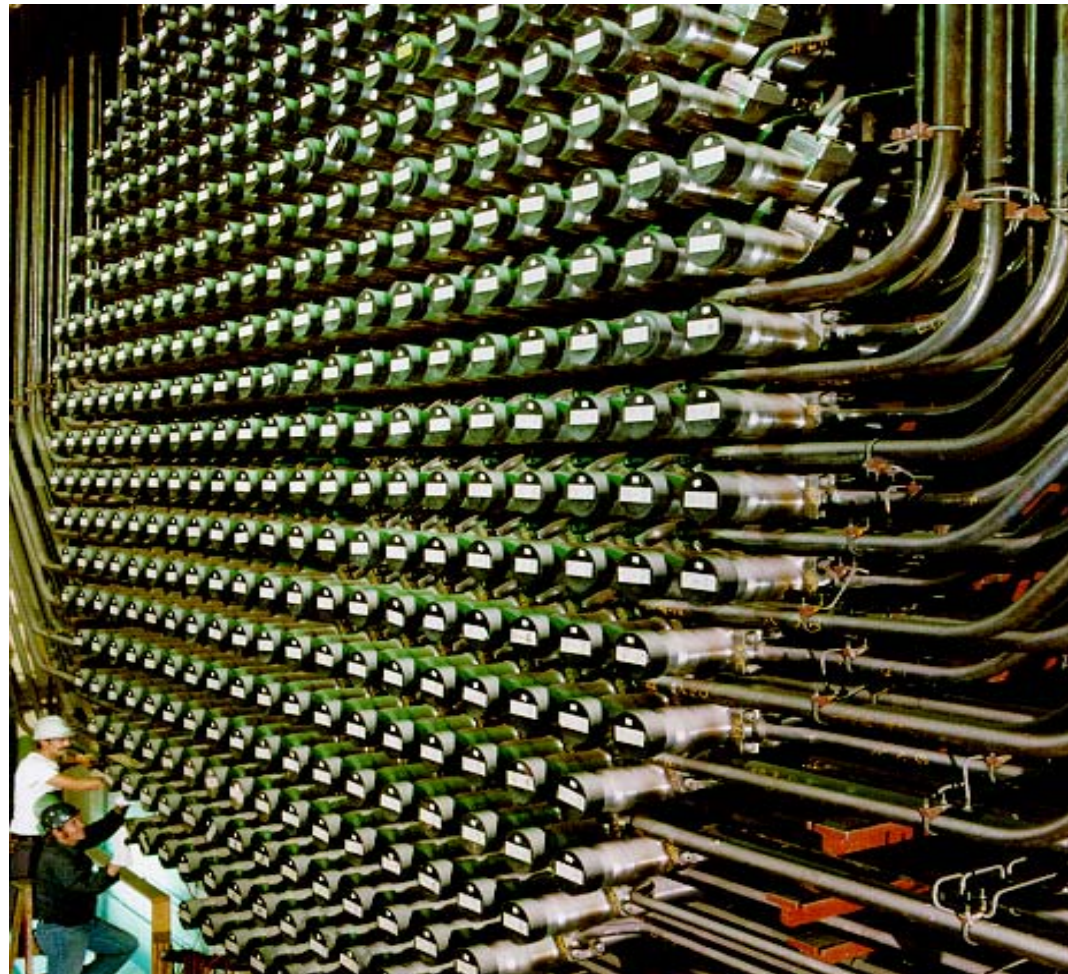


- **cracks intergranular**
- **medium-to-high-flow channels**
- **areas of high residual stress (many tight bends close to reactor face – “warm bent”)**



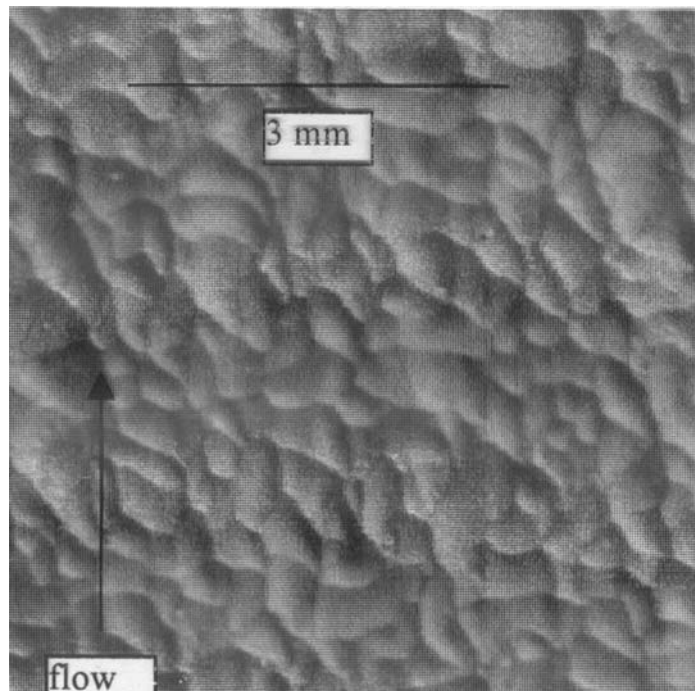


## END-FITTINGS AND FEEDERS

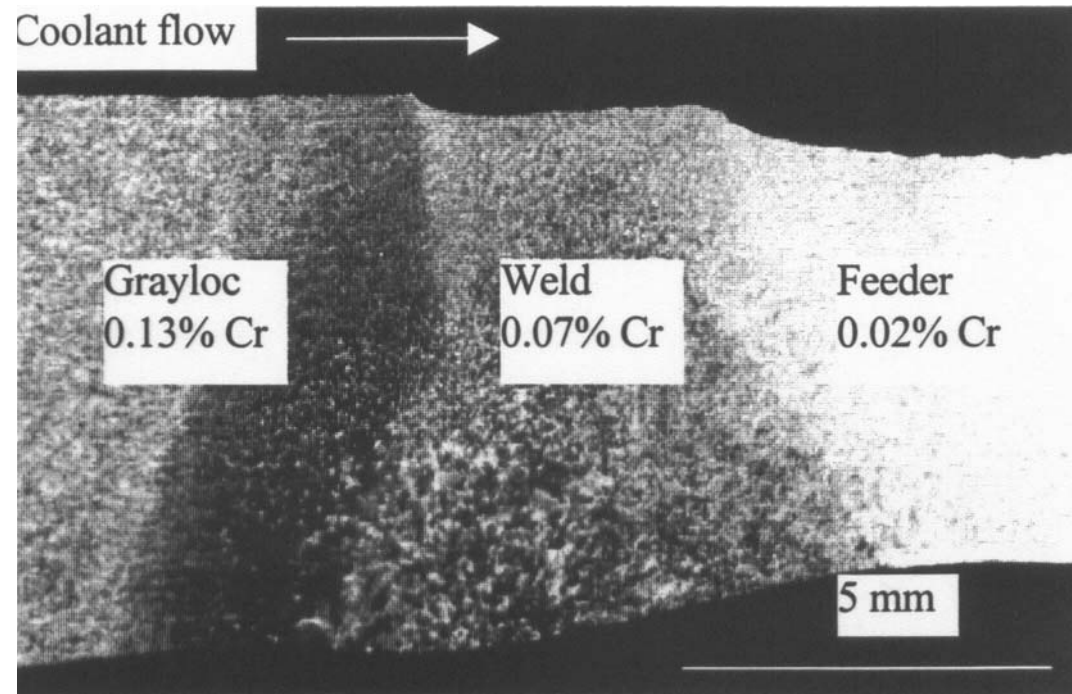


## DETAILS OF REMOVED FEEDER INDICATE FAC

**Scallops**



**Effects of Cr content of steel**

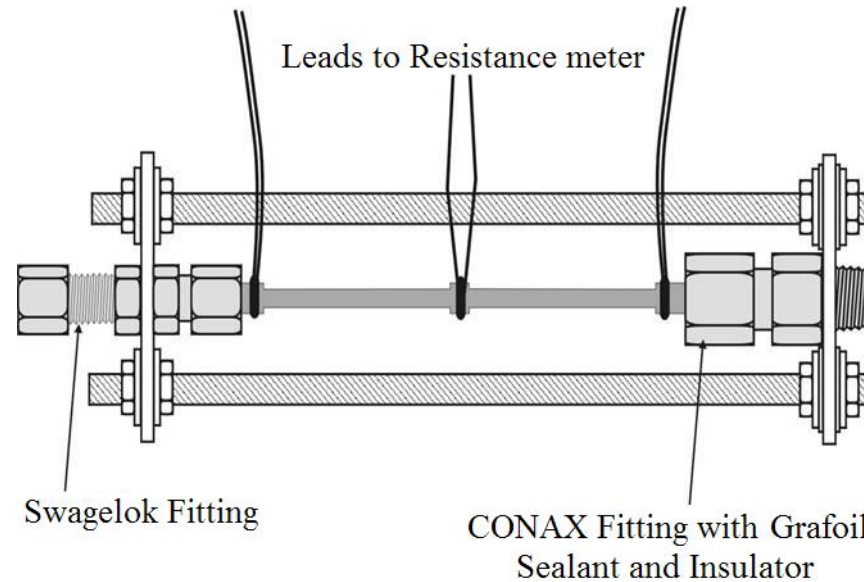


**Low-Cr steel a consequence of specifying low-Co material to control activity transport.**

- **All pre-Qinshan CANDUs display FAC: managed by stress analysis, dispositioning, removal and replacement if necessary;**
- **Darlington plant particularly susceptible (has thinner-wall feeders);**
- **Following a suggestion and early work at UNB Nuclear and later qualification studies, Darlington injected  $\text{TiO}_2$  as possible inhibitor into one channel at Unit 3 in 2004 – observed a fall in FAC of > 25%: promising;**
- **Still too many uncertainties to proceed with Ti;**
- **Qinshan expected to have low FAC rates because 0.3% Cr steel specified.**

## FAC EXPERIMENTS AT UNB

**Developed probe for on-line measurement of FAC in high-temperature water loop:**



**Increase in electrical resistance of thinning tube reflects FAC – “resistance probe”.**

**Probes made from Point Lepreau steel: SA106 Grade B (0.019%Cr)**

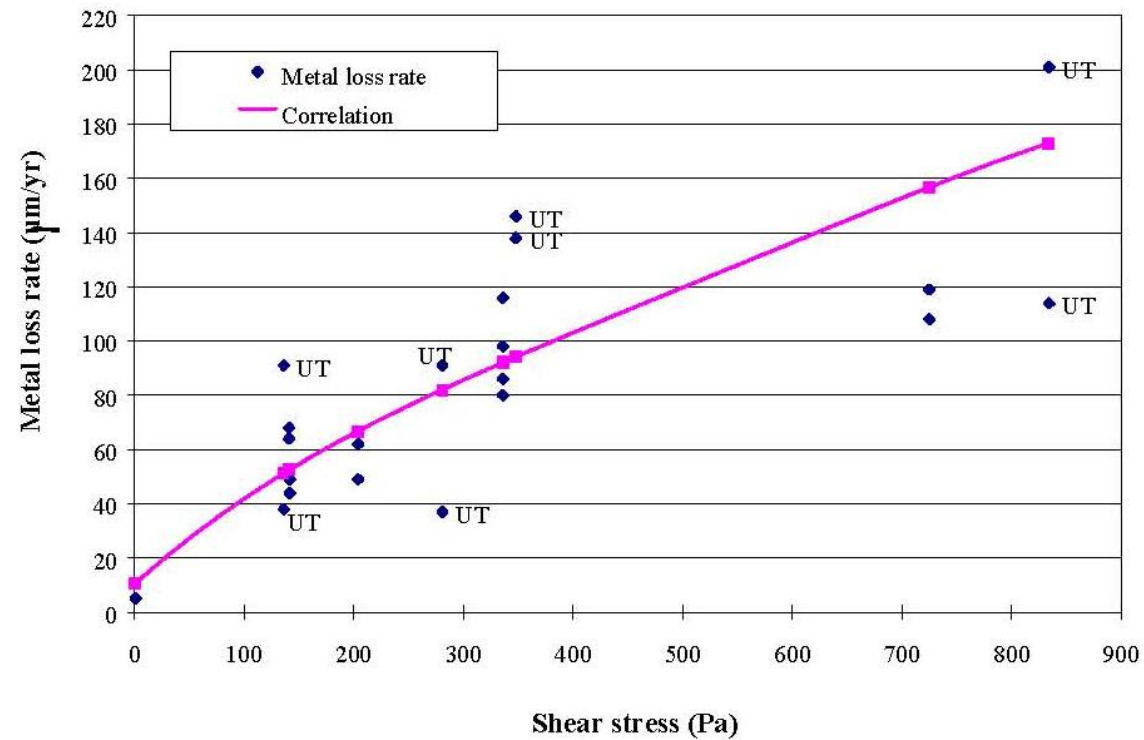


**Also developed wire probe – on-line measurement of resistance of thinning wire reflects FAC.**

**Experiments performed over range of conditions:**

- **flow rate;**
- **pH (LiOH);**
- **[Fe] saturation.**

## Scatter of results high .....



**These results and plant data suggest shear stress important (flow dependence high).**

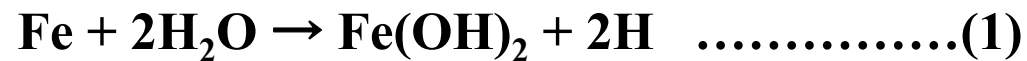
## TENTATIVE CORRELATION DERIVED:

$$CR = 7.65 (1 + 0.111 \tau^{0.75}) \times (1 + 1.51 \times 10^{-9} e^{1.87\text{pH}})$$

- Predicts reducing pH from 10.6 to 10.2 will decrease FAC rate by only ~ 12%; differences among plants indicate no consistent trend;
- Predicts flow-rate dependence similar to plant.

**N.B.** Experiments with coolant nominally saturated in [Fe] still produced FAC – if at a lower rate

**FAC processes at M-O produce H<sub>2</sub>**



**Molecular hydrogen from (2) diffuses to bulk coolant.**

**Atomic hydrogen from (1) effuses quantitatively through metal  
– measurement reflects FAC.**

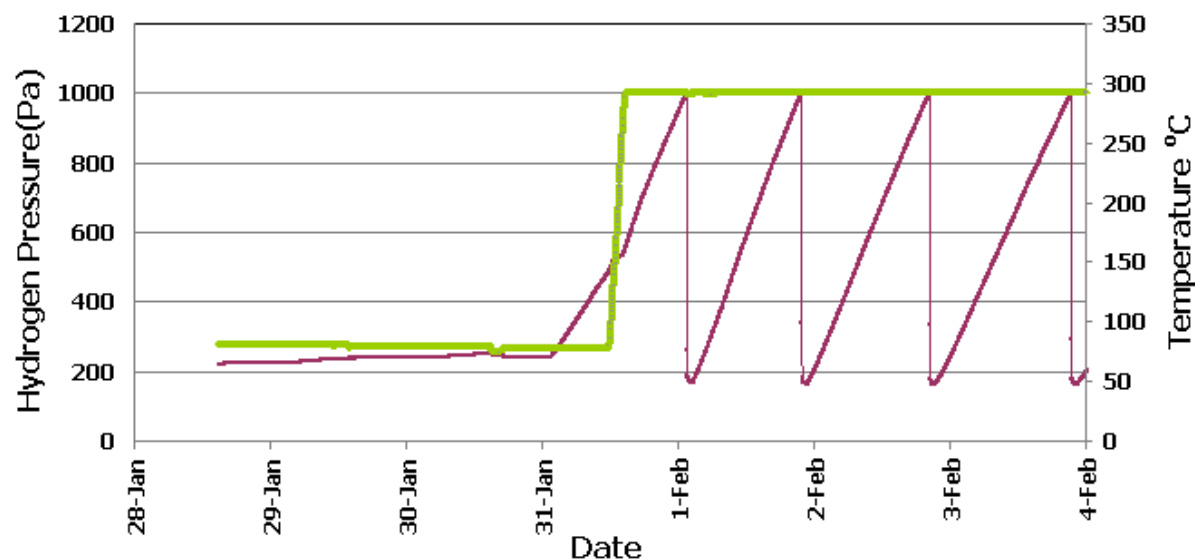


## “HYDROGEN EFFUSION PROBE” - HePro®

Collection cup clamped to outside of pipe – evacuated periodically – increase in pressure monitored.



Diagram of HePro®  
mounted on pipe



Typical pressure  
increase

## INDUSTRIAL APPLICATIONS



**Feeder at Point  
Lepreau**

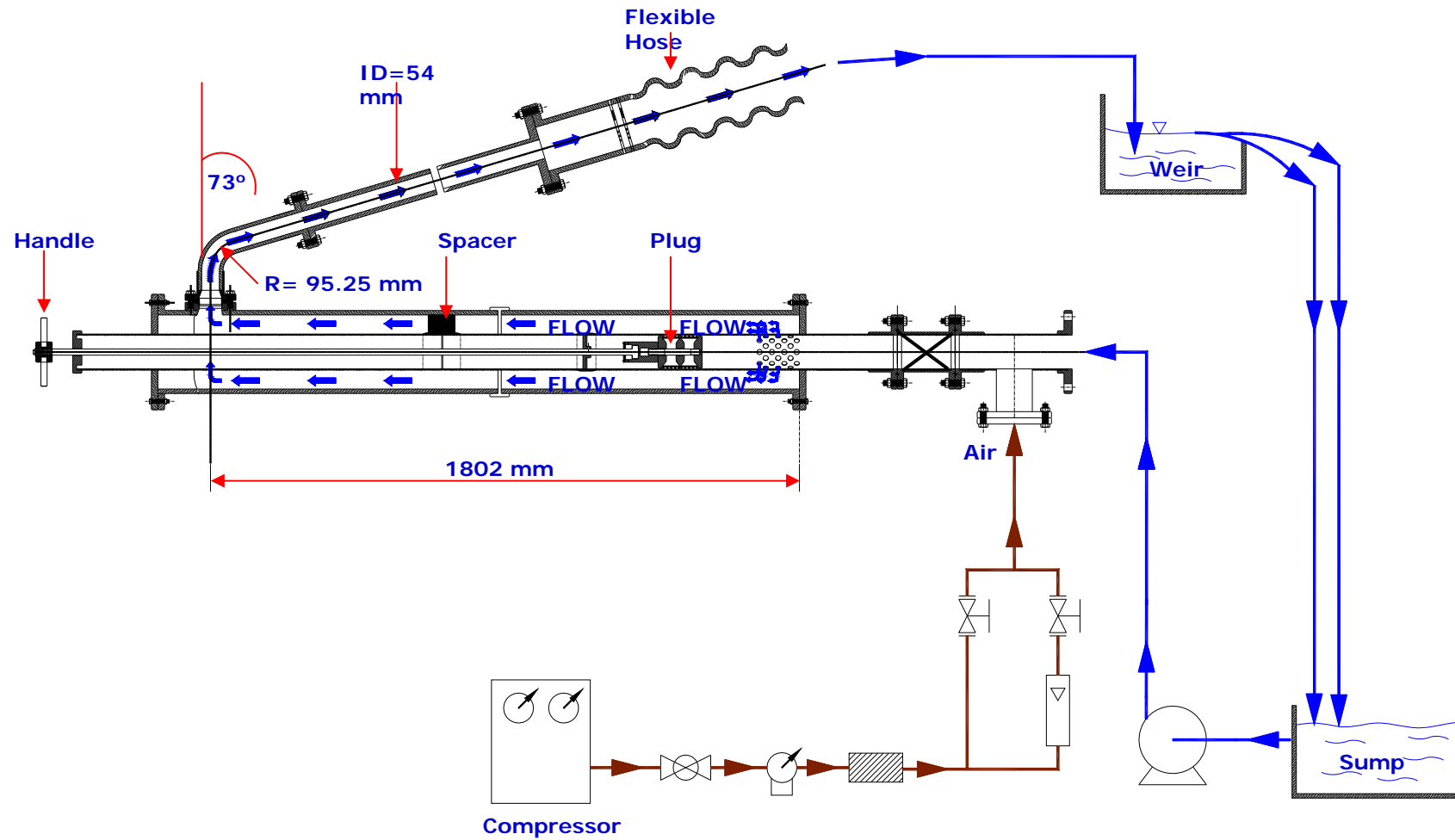


**Water wall at  
Coleson-Cove oil-  
fired plant**

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## THERMALHYDRAULIC STUDY AT UNB

- **High velocity dependence of FAC suggests shear stress distribution important;**
- **Full-scale mock-up of feeder and end-fitting constructed from transparent acrylic;**
- **Mounted in high-flow water circuit, two-phase (air-water)  $\Delta P$  and phase distribution studied at  $\sim 25^\circ \text{ C}$ ;**
- **Computer simulations with cfd code FLUENT tested against observations.**



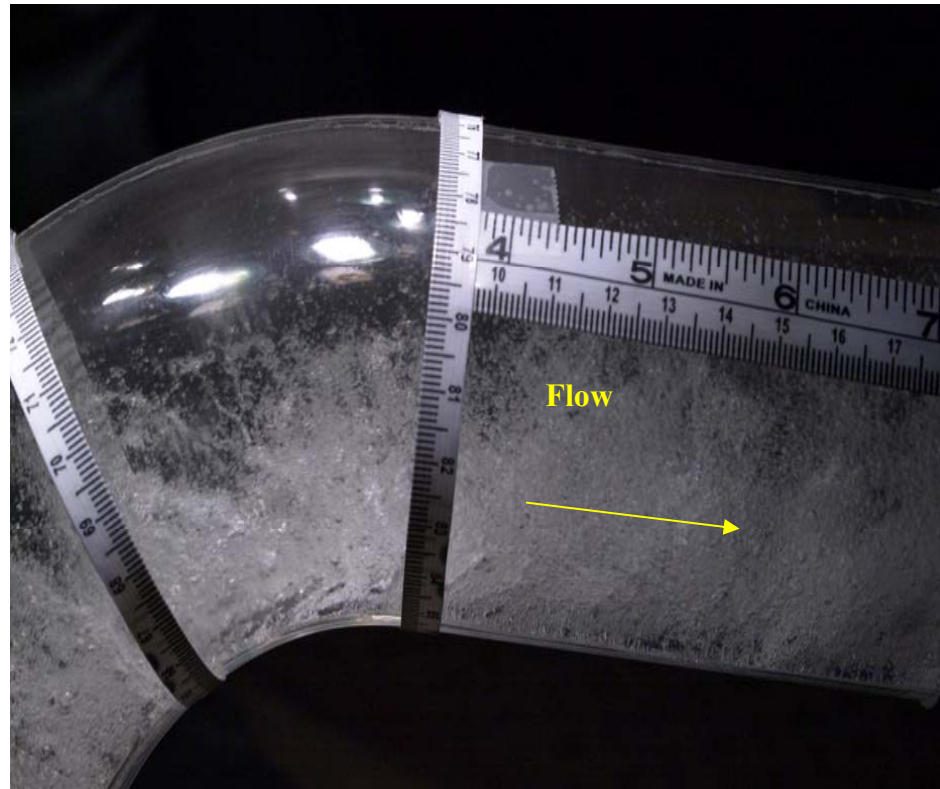
Hydraulic loop for testing feeder

## PLANT AND EXPERIMENTAL CONDITIONS

	<u>CANDU-6</u>	<u>experiment</u>
coolant	D <sub>2</sub> O	H <sub>2</sub> O
temperature (° C)	310	25
chemistry (pH <sub>25° C</sub> )	10.2 (Li)	neutral
voidage	0~0.3 (steam)	0~0.5 (air)
Re <sub>liq</sub>	~4.3x10 <sup>6</sup>	5x10 <sup>5</sup>

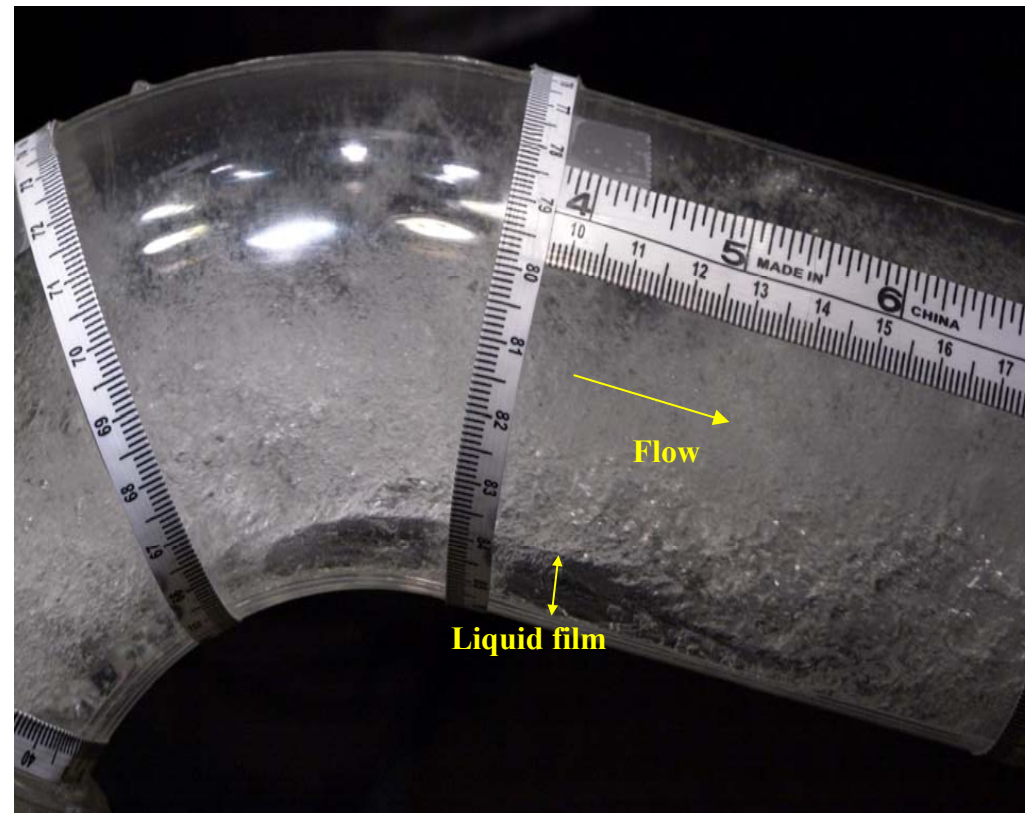


## PREDICTABLE FLOW PATTERN IN BEND



**0.05 voidage,  $Re_{liq} \sim 4.5 \times 10^5$**

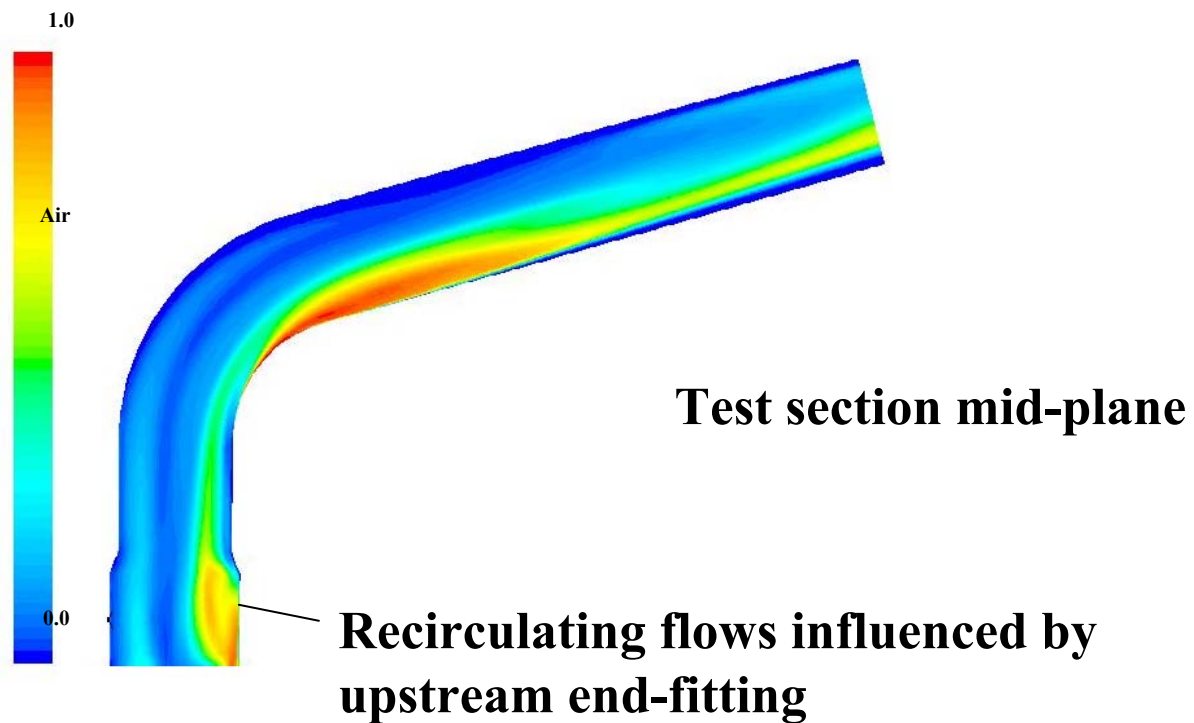
## UNPREDICTABLE FLOW PATTERN IN BEND



0.2 voidage,  $Re_{liq} \sim 4.2 \times 10^5$



## FLUENT FAILS TO PREDICT PHASE DISTRIBUTION IN BEND



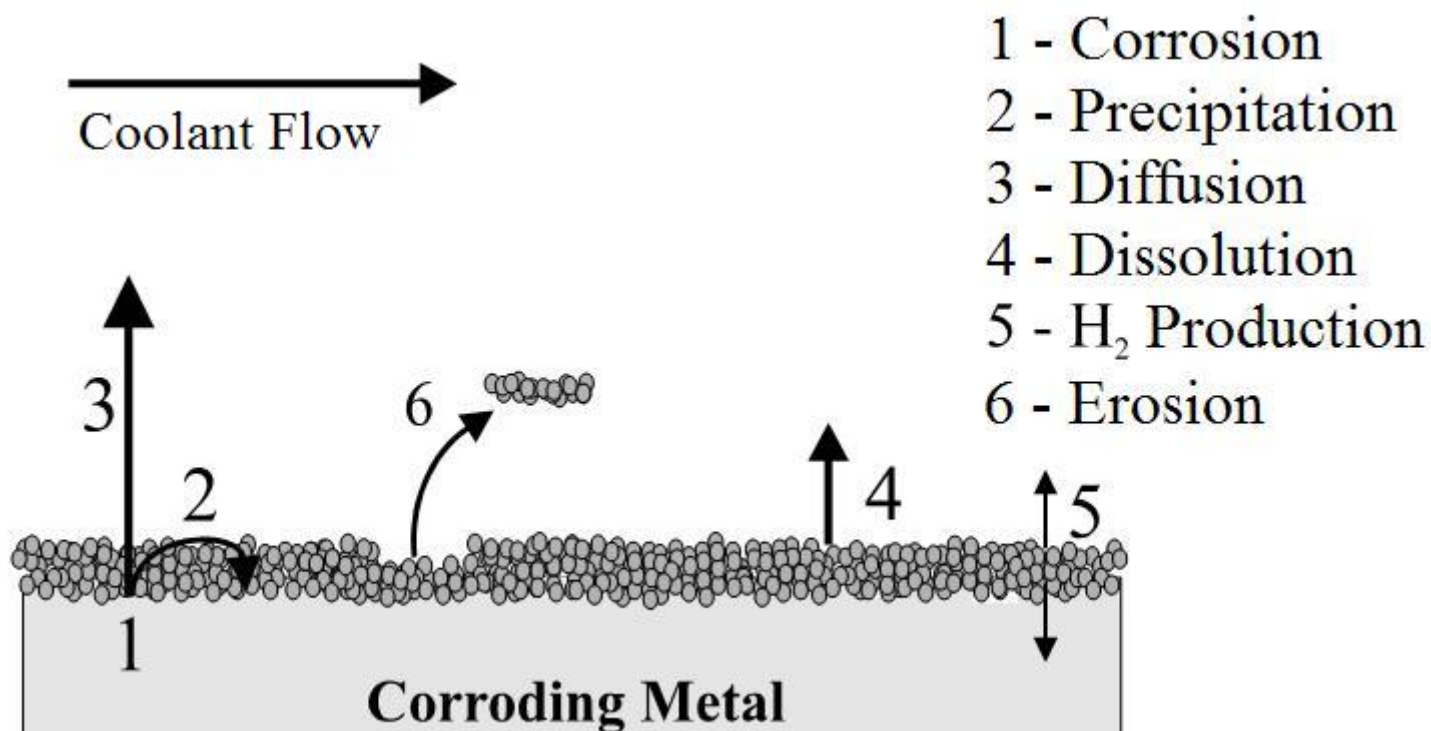
0.2 voidage,  $Re_{liq} \sim 4.2 \times 10^5$

## **SCALLOPING**

- **Effect of surface sculpting as “scallop” still unknown;**
- **Research at UNB Nuclear indicates scallops cannot be treated as conventional “sand-grain roughness” (i.e., influencing  $\Delta P$  via Moody relations);**
- **Scallop shapes create different  $\Delta P$  in forward and reverse flow;**
- **Experiments under feedwater conditions (FAC at 140° C in neutral water) indicate constant “scallop Reynolds number”.**



## PROCESSES TO BE MODELLED



## MODEL REFLECTS SYNERGY BETWEEN FILM DISSOLUTION AND SPALLING

- Electrochemical processes affected by potential – ECP computed in parallel;
- Diffusing  $\text{Fe}^{2+}$  predicted to be unchanged  $\text{Fe}(\text{OH})_2$  – straightforward Fick's Law applied;
- Rate constant for magnetite dissolution from literature (one reference) **SHOULD CONTROL** ( $\ll$  m.t.c.);
- Same rate constant assumed for precipitation (no reference);
- Erosion treated stochastically – random-number generator used to decide size of particle to spall from size distribution;
- Time for particle to be loosened and removed;

$$\theta \propto \frac{d}{\tau \cdot Q_i k_d \Delta C}$$

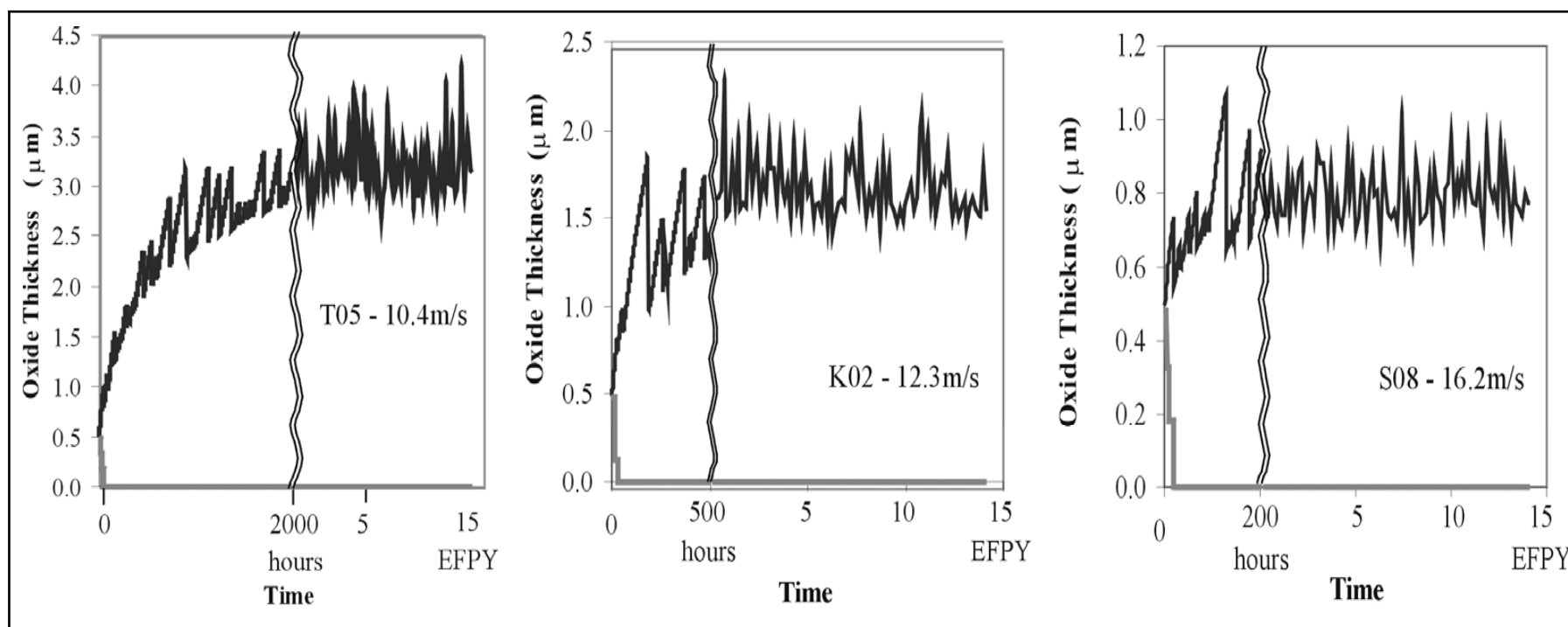
- When particle disappears, film thins instantaneously by  $d$  and corrosion rate jumps.

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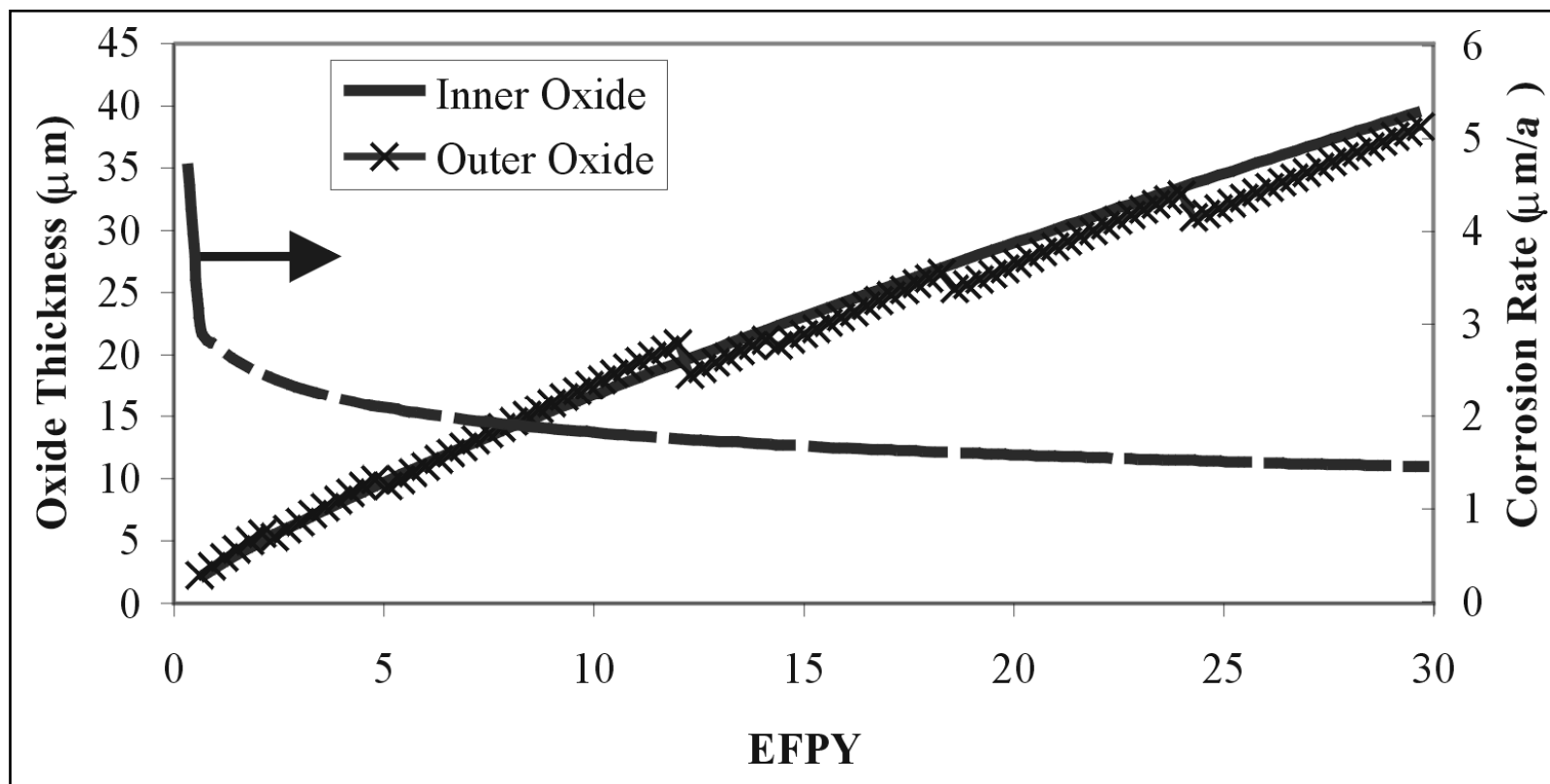
## **MODEL PREDICTS (using CFD code to calculate local shear stress)**

- **corrosion rates at different positions in feeders;**
- **ECP  $\sim$  -700 to -900 mV (vs. SHE);**
- **oxide film thicknesses (0.7  $\mu\text{m}$  to 3  $\mu\text{m}$ );**
- **growth of oxide to steady-state;**
- **corrosion and oxide film thickness of inlet feeders (saturated in dissolved iron, lower temperature).**

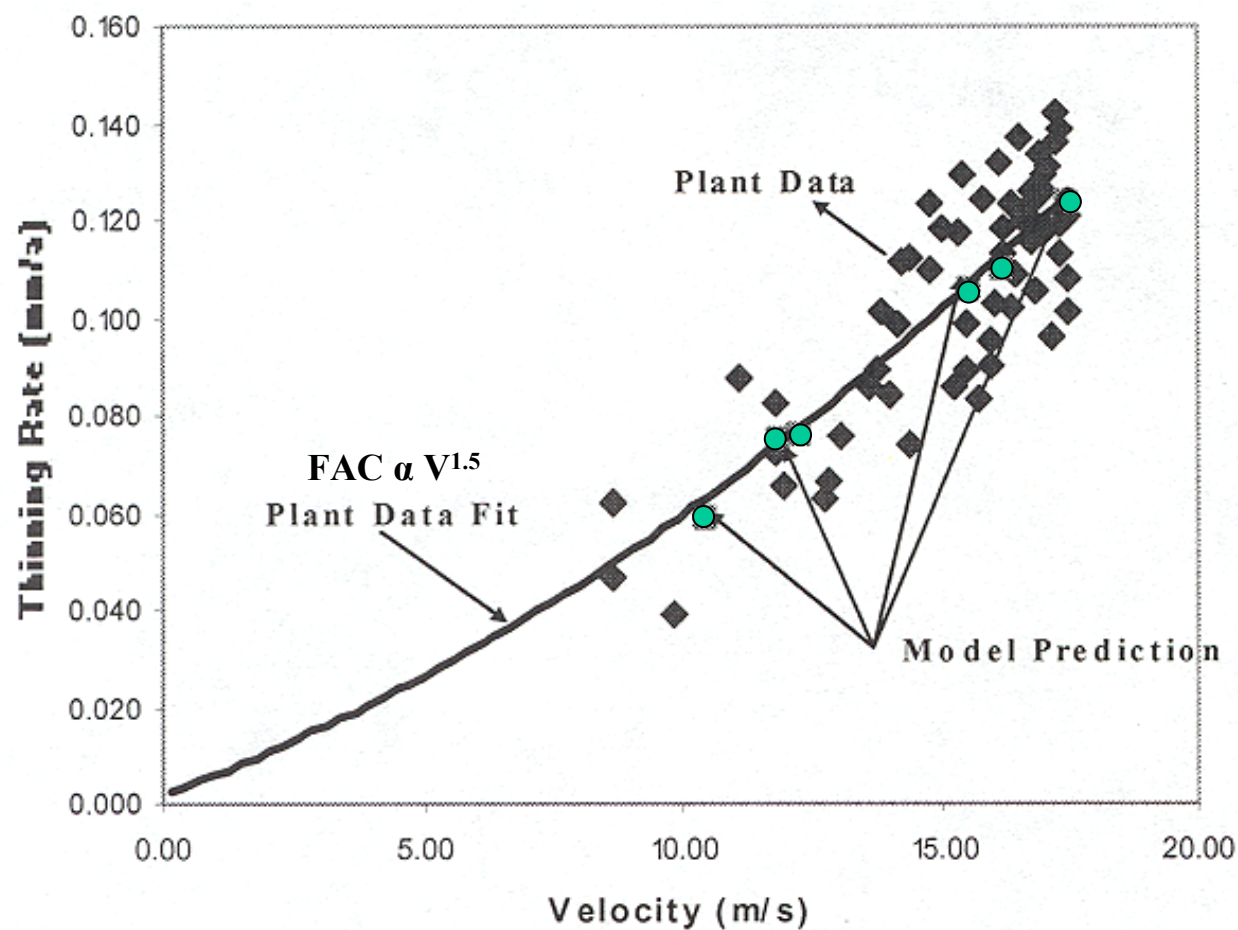
## PREDICTED OXIDE FILM THICKNESSES IN OUTLET FEEDERS



## PREDICTED CORROSION RATE AND OXIDE FILM THICKNESS FOR INLET FEEDER:



## MODEL FITS PLANT DATA WELL





## **ACKNOWLEDGEMENT**

**Many people at UNB Nuclear, including students passing through and graduating, have contributed to the findings described in this talk; my thanks are due to them.**

**The Centre for Nuclear Energy Research (CNER) at UNB is thanked for ongoing collaboration.**

**Finally, I am grateful to the CANDU Owners Group (COG) and its partners and the Natural Science and Engineering Research Council (NSERC) of Canada for continuing support for UNB Nuclear.**